NEXTera® ENERGY

POINT BEACH

December 28, 2015

NRC 2015-0077
10 CFR 50.73

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Unit 1
Docket 50-266
Renewed License No. DPR-24

Licensee Event Report 266/2015-006-01
Unit 1 Automatic Reactor Trip - Revision

Enclosed is Licensee Event Report (LER) 266/2015-006-01 for Point Beach Nuclear Plant, Unit 1. NextEra Energy Point Beach, LLC is providing this revised LER regarding the Unit 1 automatic reactor trip.

This letter contains no new regulatory commitments.

If you have any questions please contact Mr. Bryan Woyak, Licensing Manager, at 920/755-7599.

Very truly yours,

NextEra Energy Point Beach, LLC

Eric McCartney
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW

NRC FORM 366 (02-2014) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104 EXPIRES: 01/31/2017

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See Page 2 for required number of digits/characters for each block)

1. FACILITY NAME	2. DOCKET NUMBER	3. PAGE
Point Beach Nuclear Plant Unit 1	05000266	1 OF 2

4. TITLE

Unit 1 Automatic Reactor Trip

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	Rev NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
									NA	NA
11	28	2015	2015	006	01	12	28	2015	FACILITY NAME	DOCKET NUMBER
									NA	NA

9. OPERATING MODE	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: *(Check all that apply)*			
MODE 1	☐ 20.2201(b)	☐ 20.2203(a)(3)(i)	☐ 50.73(a)(2)(i)(C)	☐ 50.73(a)(2)(vii)
	☐ 20.2201(d)	☐ 20.2203(a)(3)(ii)	☐ 50.73(a)(2)(ii)(A)	☐ 50.73(a)(2)(viii)(A)
	☐ 20.2203(a)(1)	☐ 20.2203(a)(4)	☐ 50.73(a)(2)(ii)(B)	☐ 50.73(a)(2)(viii)(B)
	☐ 20.2203(a)(2)(i)	☐ 50.36(c)(1)(i)(A)	☐ 50.73(a)(2)(iii)	☐ 50.73(a)(2)(ix)(A)
10. POWER LEVEL	☐ 20.2203(a)(2)(ii)	☐ 50.36(c)(1)(ii)(A)	☒ 50.73(a)(2)(iv)(A)	☐ 50.73(a)(2)(x)
100%	☐ 20.2203(a)(2)(iii)	☐ 50.36(c)(2)	☐ 50.73(a)(2)(v)(A)	☐ 73.71 (a)(4)
	☐ 20.2203(a)(2)(iv)	☐ 50.46(a)(3)(ii)	☐ 50.73(a)(2)(v)(B)	☐ 73.71 (a)(5)
	☐ 20.2203(a)(2)(v)	☐ 50.73(a)(2)(i)(A)	☐ 50.73(a)(2)(v)(C)	☐ OTHER
	☐ 20.2203(a)(2)(vi)	☐ 50.73(a)(2)(i)(B)	☐ 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME	TELEPHONE NUMBER (Include Area Code)
Thomas P. Schneider, Senior Licensing Engineer	920-755-7797

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
B	EL	TD	C770	Y	NA	NA	NA	NA	NA

14. SUPPLEMENTAL REPORT EXPECTED	15. EXPECTED SUBMISSSION DATE	MONTH	DAY	YEAR
YES (*If yes, complete 15. EXPECTED SUBMISSION DATE*) X NO		NA	NA	NA

ABSTRACT *(Limit to 1400 spaces i.e, approximately 15 single-spaced typewritten lines)*

On November 28, 2015 with Unit 1 in MODE 1 at full power, an automatic reactor trip was actuated as the result of a failure of the main generator automatic voltage regulator (AVR). All control rods fully inserted into the core. The Auxiliary Feedwater Pumps started as expected on low steam generator level. The Auxiliary Feedwater system was secured and decay heat removal was provided by the Condenser Steam Dumps utilizing the Main Feedwater system.

This event is being reported pursuant to 10 CFR 50.73(a)(2)(iv)(A) for the automatic actuation of the reactor protection system and auxiliary feedwater system.

1. FACILITY NAME	2. DOCKET	6. LER NUMER			3. PAGE
		YEAR	SEQUENTIAL NUMBER	REV NO.	
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NARRATIVE

Description of the Event:

At 1912 on November 28, 2015 with Unit 1 in MODE 1 at full power, an automatic reactor trip was actuated as the result of a failure of the Main Generator Automatic Voltage Regulator (AVR). All control rods fully inserted into the core. The Auxiliary Feedwater Pumps started as expected on low steam generator level. The Auxiliary Feedwater system was secured and decay heat removal was provided by the Condenser Steam Dumps utilizing the Main Feedwater system. All other safety systems functioned as designed.

The Unit 1 Main Generator AVR was repaired, returned to service and the unit returned to full power operations.

This 60-day licensee event report is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(iv)(A) for the automatic actuation of the reactor protection system and auxiliary feedwater system.

Cause of the Event:

The cause of the automatic reactor trip was due to the failure of an isolation transducer module in the Main Generator AVR.

Analysis of the Event:

Troubleshooting activities completed as part of the post trip investigation identified a failed isolation transducer module within the main generator AVR. The main generator AVR isolation transducer module failure resulted in an indicated over excitation protection set point being exceeded causing a generator lockout. The generator lockout caused the turbine to trip. The reactor tripped as a result of the turbine trip.

Corrective Actions:

The Unit 1 Main Generator AVR was repaired, returned to service and the unit returned to full power operations.

Safety Significance:

The event was determined to be of very low safety significance. During the event and subsequent recovery actions, there was no loss of any safety systems, structures or components. All control rods fully inserted into the core as designed to control reactivity and temperature of the core. This event had no impact on the safety of the core. The Auxiliary Feedwater Pumps started as expected on low steam generator level. The Auxiliary Feedwater system was secured and decay heat removal was provided by the Condenser Steam Dumps utilizing the Main Feedwater system. There was no impact on the health and safety of the public as a result of this event.

Similar Events:

There have not been similar events of automatic reactor trips in the past three years.

Component Failure Data:

Main Generator Automatic Voltage Regulator DC-DC Isolation Transducer Module Style 1A96166G11 – Cutler-Hammer